ELSEVIER

Nuclear Engineering and Design 159 (1995) 277–278

Nuclear Engineering and Design

Author Index of Volume 159

Ahlstrand, R., see Wallin, K. 159 (1995) 69
Assice, A., see D'Annucci, F. 159 (1995) 177

Bartonicek, J., see Seibold, A. 159 (1995) 1
Bäro, G., see D'Annucci, F. 159 (1995) 177
Berg, H.P. and D. Gründler, Modelling of radioactive material release from waste packages in case of a fire event 159 (1995) 217
Bergner, F., see Böhmert, J. 159 (1995) 131
Blom, K.-H., W. Krull and K.-R. Levknecht, Information and signalling systems for FRG-1 and FRG-2 159 (1995) 153
Blumer, U., see Lucht, B. 159 (1995) 91
Böhmert, J., F. Bergner, M. Grosse and H.-W. Viehrig, Influence of the depth position on the neutron embrittlement of the VVER reactor pressure vessel steel 15Cr2MoV(A) Consequences for the assessment of reactor safety 159 (1995) 131

D'Annucci, F., G. Bäro and A. Assice, Inspections of pressure vessel head penetrations 159 (1995) 177
Degen, G., J. Mertens and B. Reer, ESAP, an easy-to-use expert system for the systematic analysis of operator actions within the scope of probabilistic risk assessment 159 (1995) 259
Diersch, R., N. Ulrich, R. Hüggenberg, K. Janberg and A. Ettemeyer, Experimental investigations on a fuel element transportation vessel for analysis of mechanical loads 159 (1995) 197
Dubourg, M., Hard chemical decontamination of steam generator tube bundles 159 (1995) 123

Ettemeyer, A., see Diersch, R. 159 (1995) 197

Fehér, S., see Valkó, J. 159 (1995) 225
Feuser, W., Investigations concerning the semihydrostatic emplacement of waste packages in vertical boreholes 159 (1995) 239

Gavrilov, S. and V. Kremnev, Decommissioning of nuclear installations in Russia and the other countries of the CIS 159 (1995) 105
Gompper, K., see Kunze, S. 159 (1995) 209
Gortnar, O., see Petelin, S. 159 (1995) 169
Grosse, M., see Böhmert, J. 159 (1995) 131
Gründler, D., see Berg, H.P. 159 (1995) 217

Hauser, W., W. Wenzel and R. Müller, PROMAX/PC—computer code for computing the maximum activity of radioactive waste packages and design of gamma-shielding 159 (1995) 233
Hieronymus, W., see Löscher, B. 159 (1995) 163
Hoogenboom, J.E., see Valkó, J. 159 (1995) 225
Humair, D., see Lucht, B. 159 (1995) 91
Hüggenberg, R., see Diersch, R. 159 (1995) 197
Hüttel, G. and C. Reiche, Investigation in the zero power reactor RAKE for an effective neutronic coupling of water-surrounded beam tubes 159 (1995) 213

Janberg, K., see Diersch, R. 159 (1995) 197

Kockelmann, H., see Seibold, A. 159 (1995) 1
Krell, J., B. Mängel and H. Rebohm, Analysis of rod-drop measurements at a VVER-440 reactor 159 (1995) 265
Kremnev, V., see Gavrilov, S. 159 (1995) 105
Krull, W., see Blom, K.-H. 159 (1995) 153
Kunze, S. and K. Gompper, Drying medium level liquid waste concentrates 159 (1995) 209

Levknecht, K.-R., see Blom, K.-H. 159 (1995) 153
Liu, X., see Sievers, J. 159 (1995) 63
Lorenz, O., see Seepolt, R. 159 (1995) 183
Löscher, B., H. Quaas and W. Hieronymus, Mobile system for the management of fuel assemblies from research reactors 159 (1995) 163
Löschhorn, U., see Volkmann, B. 159 (1995) 117
Lucht, B., D. Humair and U. Blumer, Fatigue monitoring and life cycle analysis of a recuperative heat exchanger in the primary circuit 159 (1995) 91

Markgraf, J.F.W., G.P. Tartaglia and G. Tsotridis, Irradiation devices and irradiation programmes at the high flux reactor (HFR) Petten for the investigation of the irradiation behaviour of stainless steels 159 (1995) 81
Mavko, B., see Parzer, I. 159 (1995) 143
Mavko, B., see Petelin, S. 159 (1995) 169
Mängel, B., see Krell, J. 159 (1995) 265
Mertens, J., see Degen, G. 159 (1995) 259
Müller, R., see Hauser, W. 159 (1995) 233

Parzer, I., S. Petelin and B. Mavko, Modelling operator rediagnosis during an SGTR event 159 (1995) 143
Petelin, S., see Parzer, I. 159 (1995) 143
Petelin, S., B. Mavko and O. Gortnar, Analysis of operator actions during the accident caused by total loss of secondary heat sink 159 (1995) 169

Quaas, H., see Löscher, B. 159 (1995) 163

Raiko, H., see Sievers, J. 159 (1995) 63
Rajamäki, P., see Sievers, J. 159 (1995) 63
Rebohm, H., see Krell, J. 159 (1995) 265
Reer, B., see Degen, G. 159 (1995) 259
Reiche, C., see Hüttel, G. 159 (1995) 213
Rintamaa, R., see Wallin, K. 159 (1995) 69

Schäfer, L., Estimating the tensile properties of an austenitic cladding tube steel following corrosion by uranium dioxide, simultated fission products and neutron exposure 159 (1995) 55
Schäfer, L., Estimating the tensile properties of an austenitic cladding tube steel following corrosion by uranium dioxide, simultated fission products and neutron exposure 159 (1995) 29
Schäfer, L., Estimating the tensile properties of an austenitic cladding tube steel following corrosion by uranium dioxide, simultated fission products and neutron exposure 159 (1995) 43
Seepolt, R. and O. Lorenz, Surveillance of nuclear-thermohydraulic stability in BWR operation 159 (1995) 183
Seibold, A., J. Bartonicek and H. Kockelmann, Operational monitoring in German nuclear power plants 159 (1995) 1
Seifert, E., Efficiency of Monte Carlo perturbation calculations 159 (1995) 203
Sievers, J., X. Liu, P., Rajamäki, H. Talja and H. Raiko, Comparative analyses concerning integrity of a VVER-440 reactor pressure vessel 159 (1995) 63
Slobben, J., see Valkó, J. 159 (1995) 225

Talja, H., see Sievers, J. 159 (1995) 63
Tartaglia, G.P., see Markgraf, J.F.W. 159 (1995) 81
Törrönen, K., see Wallin, K. 159 (1995) 69
Tsotridis, G., see Markgraf, J.F.W. 159 (1995) 81
Tunaboylu, K., Safety requirements for the design of large containers for decommissioning waste 159 (1995) 189

Ulrich, N., see Diersch, R. 159 (1995) 197

Valkó, J., S. Fehér, J.E. Hoogenboom and J. Slobben, Calculation of the void reactivity of CANDU lattices using the SCALE code system 159 (1995) 225
Valo, M., see Wallin, K. 159 (1995) 69
Viehrig, H.-W., see Böhmert, J. 159 (1995) 131
Volkmann, B. and U. Löschhorn, Aspects on decommissioning of the Greifswald nuclear power plant 159 (1995) 117

Wallin, K., M. Valo, R. Rintamaa, K. Törrönen and R. Ahlstrand, Descriptive characteristics of different types of test for irradiation embrittlement 159 (1995) 69
Wenzel, W., see Hauser, W. 159 (1995) 233

ELSEVIER

Nuclear Engineering and Design 159 (1995) 279–280

Nuclear Engineering and Design

Subject Index of Volume 159

Thermal and fluid design

Surveillance of nuclear-thermohydraulic stability in BWR operation, R. Seepolt and O. Lorenz 159 (1995) 183
Efficiency of Monte Carlo perturbation calculations, E. Seifert 159 (1995) 203

Materials properties

Estimating the tensile properties of an austenitic cladding tube steel following corrosion by uranium dioxide, simulated fission products and neutron exposure, L. Schäfer 159 (1995) 43
Influence of the depth position on the neutron embrittlement of the VVER reactor pressure vessel steel 15Cr2MoV(A). Consequences for the assessment of reactor safety, J. Böhmert, F. Bergner, M. Grosse and H.-W. Viehrig 159 (1995) 131

Mechanical design, component fabrication and testing

Determination of limits for smallest detectable and largest subcritical leakage cracks in piping systems, R. Bieselt, B. Kuckartz and M. Wolf 159 (1995) 29
Descriptive characteristics of different types of test for irradiation embrittlement, K. Wallin, M. Valo, R. Rintamaa, K. Törrönen and R. Ahlstrand 159 (1995) 69
Fatigue monitoring and life cycle analysis of a recuperative heat exchanger in the primary circuit, B. Lucht, D. Humair and U. Blumer 159 (1995) 91
Inspections of pressure vessel head penetrations, F. D'Annucci, G. Bäro and A. Assice 159 (1995) 177

Experimental investigations on a fuel element transportation vessel for analysis of mechanical loads, R. Diersch, N. Ulrich, R. Hüggenberg, K. Janberg and A. Ettemeyer 159 (1995) 197

Reactor concepts and systems analysis

The LBB project for the new generation of WWER plants and lessions learned from the WWER 440 model 230 LBB project, J. Žd'árek, L. Pečínka and F. Kašpar 159 (1995) 55
Comparative analyses concerning integrity of a VVER-440 reactor pressure vessel, J. Sievers, X. Liu, P. Rajamäki, H. Talja and H. Raiko 159 (1995) 63
Irradiation devices and irradiation programmes at the high flux reactor (HFR) Petten for the investigation of the irradiation behaviour of stainless steels, J.F.W. Markgraf, G.P. Tartaglia and G. Tsotridis 159 (1995) 81
Analysis of operator actions during the accident caused by total loss of secondary heat sink, S. Petelin, B. Mavko and O. Gortnar 159 (1995) 169
Inspections of pressure vessel head penetrations, F. D'Annucci, G. Bäro and A. Assice 159 (1995) 177
ESAP, an easy-to-use expert system for the systematic analysis of operator actions within the scope of probabilistic risk assessment, G. Degen, J. Mertens and B. Reer 159 (1995) 259

Safeguard engineering

Modelling of radioactive material release from waste packages in case of a fire event, H.P. Berg and D. Gründler 159 (1995) 217

Information

Decommissioning of nuclear installations in Russia and the other countries of the CIS, S. Gavrilov and V. Kremnev 159 (1995) 105
Aspects on decommissioning of the Greifswald nuclear power plant, B. Volkmann and U. Löschhorn 159 (1995) 117
Hard chemical decontamination of steam generator tube bundles, M. Dubourg 159 (1995) 123

System contro' and controls

Operational monitoring in German nuclear power plants, A. Seibold, J. Bartonicek and H. Kockelmann 159 (1995) 1
Modelling operator rediagnosis during an SGTR event, I. Parzer, S. Petelin and B. Mavko 159 (1995) 143
Information and signalling systems for FRG-1 and FRG-2, K.-H. Blom, W. Krull and K.-R. Levknecht 159 (1995) 153

Nuclear waste management and disposal

Hard chemical decontamination of steam generator tube bundles, M. Dubourg 159 (1995) 123
Safety requirements for the design of large containers for decommissioning waste, K. Tunaboylu 159 (1995) 189
Drying medium level liquid waste concentrates, S. Kunze and K. Gompper 159 (1995) 209
PROMAX/PC — computer code for computing the maximum activity of radioactive waste packages and design of gamma-shielding, W. Hauser, W. Wenzel and R. Müller 159 (1995) 233
Investigations concerning the semihydrostatic emplacement of waste packages in vertical boreholes, W. Feuser 159 (1995) 239

Core physics and fuel disposal

Mobile system for the management of fuel assemblies from research reactors, B. Löscher, H. Quaas and W. Hieronymus 159 (1995) 163
Efficiency of Monte Carlo perturbation calculations, E. Seifert 159 (1995) 203
Investigations in the zero power reactor RAKE for an effective neutronic coupling of water-surrounded beam tubes, G. Hüttel and Chr. Reiche 159 (1995) 213
Calculation of the void reactivity of CANDU lattices using the SCALE code system, J. Valkó, S. Fehér, J.E. Hoogenboom and J. Slobben 159 (1995) 225
Analysis of rod-drop measurements at a VVER-440 reactor, J. Krell, B. Mängel and H. Rebohm 159 (1995) 265